

C/E ratio for spectrum averaged cross sections (SPA) in $^{235}\text{U}(\text{n}_{\text{th}},\text{f})$ field

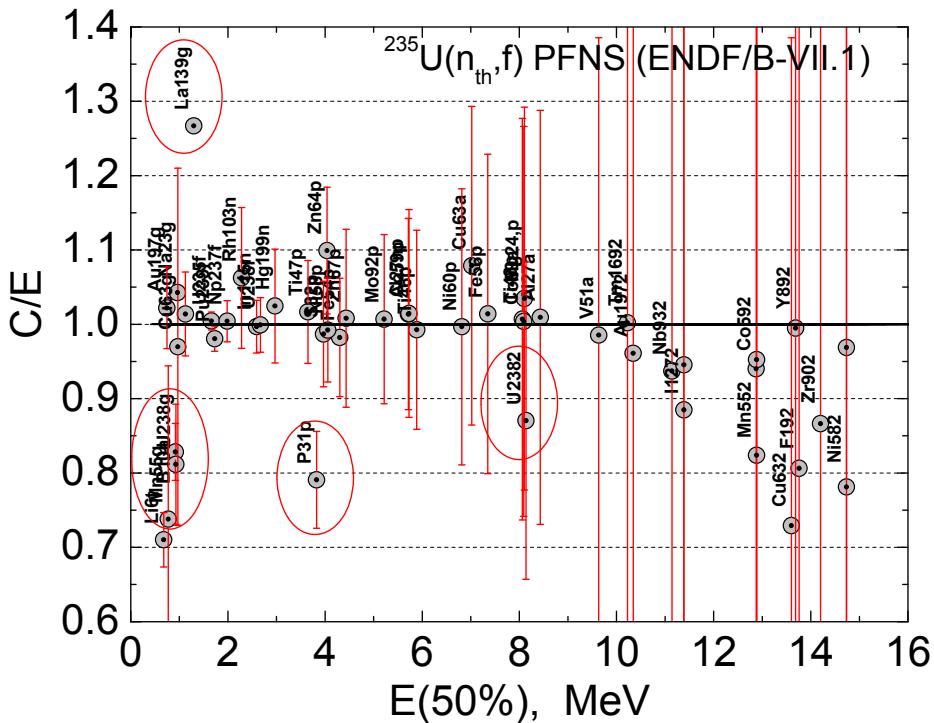


Fig. 1. C/E ratio in $^{235}\text{U}(\text{n}_{\text{th}},\text{f})$ field: obvious outliers: $^{55}\text{Mn}(\text{n},\gamma)$, $^{238}\text{U}(\text{n},\gamma)$, $^{139}\text{La}(\text{n},\gamma)$, $^{31}\text{P}(\text{n},\text{p})$ and $^{238}\text{U}(\text{n},2\text{n})$. Error bars include only experimental (black) and additionally IRDFF XS and ENDF/B-VII.1 spectrum (red) uncertainties. $^6\text{Li}(\text{n},\alpha)$, $^{10}\text{B}(\text{n},\alpha)$ are not outliers, since inclusion of other α -production reactions increase C/E up to 1.0 !

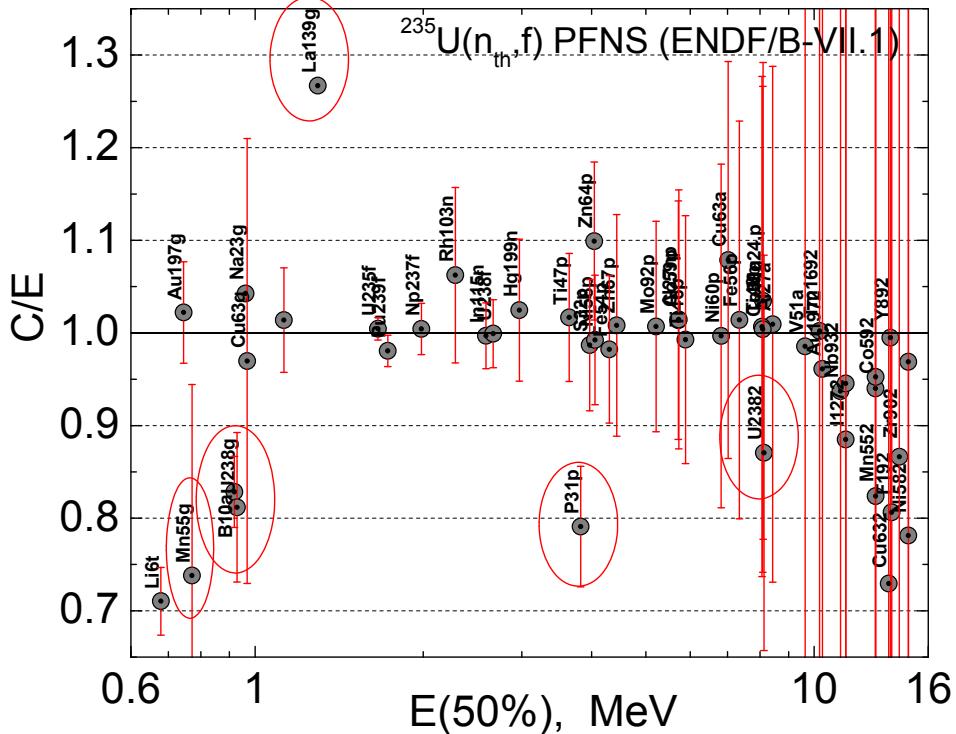


Fig. 2. The same as Fig. 1, but log scale for energy.

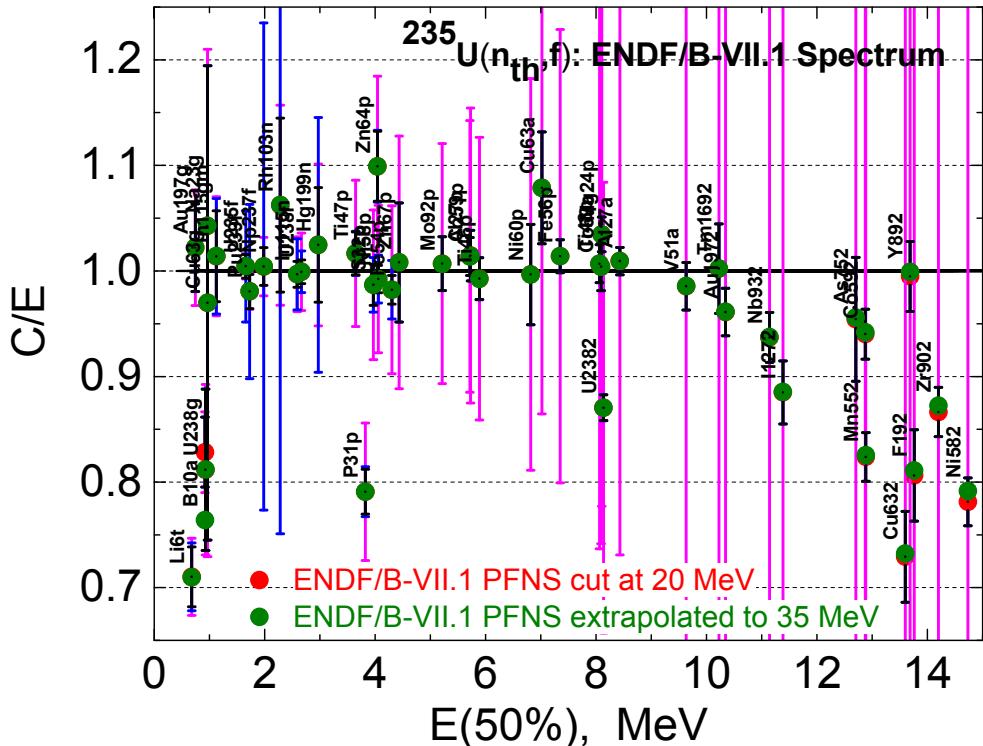


Fig. 3. C/E with IRDFF-1.03 cross sections averaged in the $^{235}\text{U}(\text{n}_{\text{th}}, \text{f})$ PFNS from ENDF/B-VII.1 [1]. Uncertainties: experimental SPA (black bars), IRDFF-1.03 cross sections (blue), evaluated spectra (pink) - not shown.

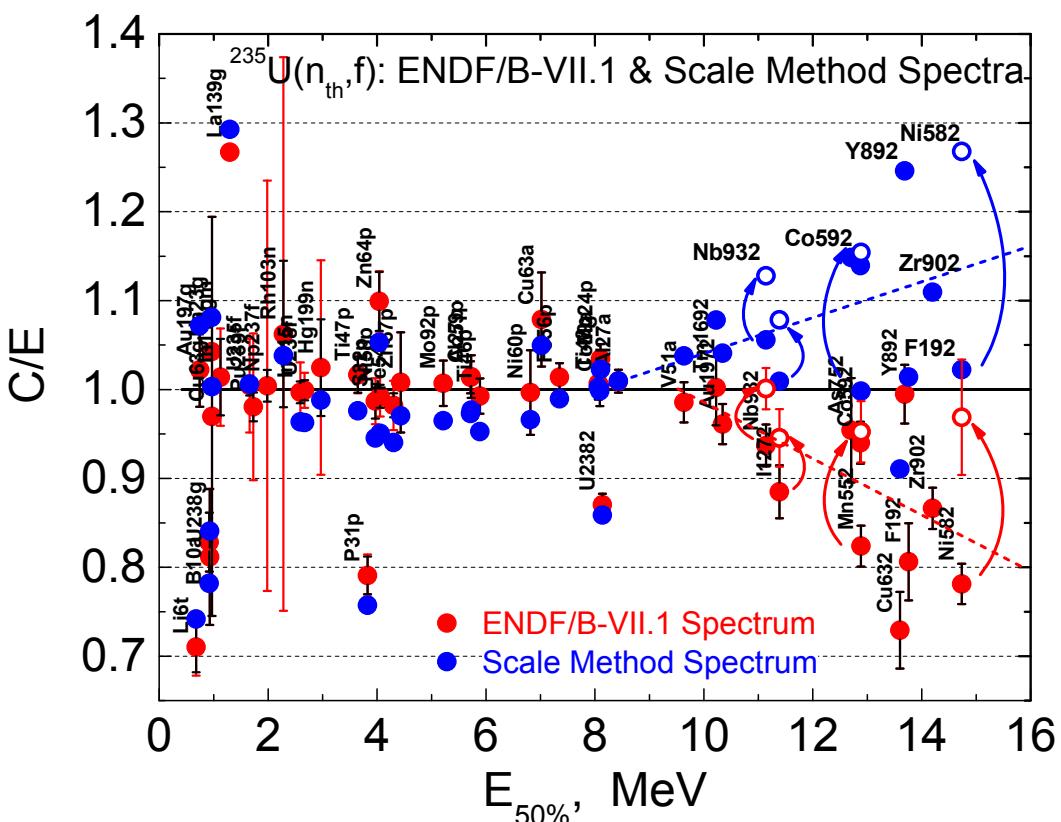


Fig. 4. C/E with IRDFF-1.03 cross sections averaged in the $^{235}\text{U}(\text{n}_{\text{th}}, \text{f})$ PFNS from ENDF/B-VII.1 [1] and Scale method [2]. Uncertainties: experimental SPA (black bars), IRDFF-1.03 cross sections (red), evaluated spectra - not shown. Three curved arrows show the change of C/E for $^{127}\text{I}(\text{n}, \text{n}_2\text{n})$, $^{55}\text{Mn}(\text{n}, \text{n}_2\text{n})$ and $^{58}\text{Ni}(\text{n}, \text{n}_2\text{n})$ when SPA recommended by W. Mannhart are replaced with K. Zolotarev values.

Reference

1. M.B. Chadwick, M. Herman et al., Nuclear Data Sheets, **112**, 2887 (2011)
2. N.V. Kornilov, Nucl. Sci. Eng., **169**, 290 (2011)

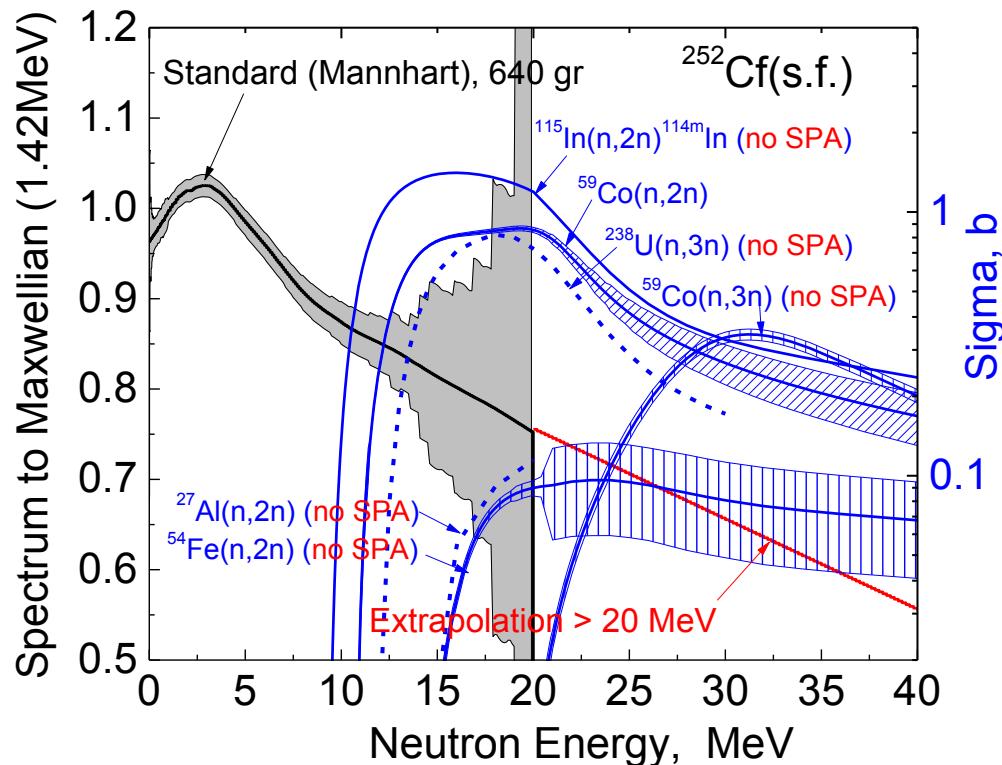
The same but for [Cf-252 field](#)

Back to [CRP web-page](#)

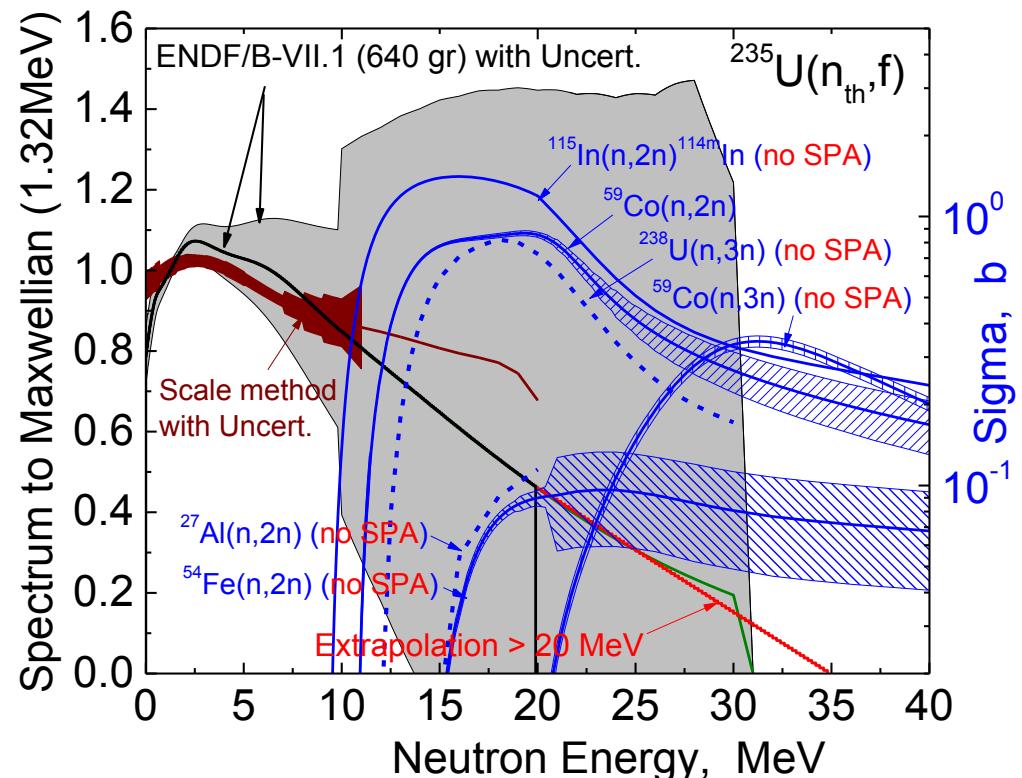
**Spectrum averaged cross sections (SPA) for the high threshold dosimetry reactions:
feasibility of activation and other alternative experimental techniques for SPA at level of 1 - 1000 μ b**

I. SPA cross sections for the high threshold dosimetry reactions

Following the recommendation of the IAEA Technical Meeting “Toward a New Evaluation of Neutron Standards”, 8-12 July 2013 ([INDC\(NDS\)- 0641](#)):
“... assessing the possibility of using the AMS technique for the measurement of the $^{235}\text{U}(n_{\text{th}},f)$ or $^{235}\text{U}(n_{\text{cold}},f)$ prompt fission neutron averaged cross sections which can be used for validation of the prompt fission neutron spectrum at energies above 8 MeV ($<\text{E}_{50\%}> > 8 \text{ MeV}$)”
the **spectrum averaged cross sections (SPA)** were calculated for several high threshold IRDFF reactions in $^{252}\text{Cf}(\text{s.f.})$ and $^{235}\text{U}(n_{\text{th}},f)$ fields:



$^{252}\text{Cf}(\text{s.f.})$ PFNS (ratio to Maxwellian T = 1.42 MeV) and IRDFF cross sections (only $^{59}\text{Co}(n,2n)$ SPA was measured).



$^{235}\text{U}(n_{\text{th}},f)$ PFNS (ratio to Maxwellian T = 1.32 MeV) and IRDFF cross sections (only $^{59}\text{Co}(n,2n)$ SPA was measured).

Table 1. Dosimetry reactions, their stable products, kinematic threshold E_{thr}, effective energy E_{50%} and SPA in the ²⁵²Cf(s.f.) field, sorted by increasing E_{50%}.

IRDFF reactions and their products	E _{thr} MeV	E _{50%} MeV	SPA, μb		N _{product} / N _{target} if $10^8 \text{n/cm}^2/\text{s}$, 1000h	Comments
			IRDFF ¹	Experiment ²		
²⁵²Cf(s.f.) Spontaneous Fission Spectra: given ²⁵²Cf produces Flux = $10^8 \text{n/cm}^2/\text{s}$ (i.e. at $\approx 1 \text{ cm}$ from ²⁵²Cf of 10^9n/s intensity³) and Irradiation of sample = 1000 h = 4.17 weeks						
⁵⁴ Fe(n, α) ⁵¹ Cr (ε , 27.7 d) \rightarrow ⁵¹ V (stable)	0	7.430	1113 \pm 3.6%	No Exp	4007 10^{-16}	
²³⁸ U(n,2n) ²³⁷ U (β^- , 6.75 d) \rightarrow ²³⁷ Np (2.14 My)	6.180	8.276	20584 \pm 2.4%	19200 \pm 10% 12200 \pm 12%	74100 10^{-16}	Blinov vs. Shani: measurements discrepant !!!
²³² Th(n,2n) ²³¹ Th (β^- , 26 d) \rightarrow ²³¹ Pa (3.28 kY)	6.448		24377 (B-VII.1)	No Exp	87757 10^{-16}	
¹⁶⁹ Tm(n,2n) ¹⁶⁸ Tm (ε , 93 d) \rightarrow ¹⁶⁸ Er (stable)	8.082	10.400	6260 \pm 2.4%	6690 \pm 6.3%	22536 10^{-16}	
¹³⁰ Te(n,2n) ¹²⁹ Te (IT, β^- , 34 d) \rightarrow ¹²⁹ I (stable)	8.484		3494 (B-VII.1)	No Exp	12578 10^{-16}	AMS threshold ^W = 10^{-14}
¹⁴¹ Pr(n,2n) ¹⁴⁰ Pr (ε , 3.4 min) \rightarrow ¹⁴⁰ Ce (stable)	9.464	11.85	1990 \pm 11.1%	No Exp.	7164 10^{-16}	
⁷⁵ As(n,2n) ⁷⁴ As (ε , 17.8 d) \rightarrow ⁷⁴ Ge (stable)	10.383	12.91	621 \pm 5.8%	No Exp.	2236 10^{-16}	
¹¹⁵ In(n,2n) ^{114m} In (IT, 50 d; β^-) \rightarrow ¹¹⁴ Sn (stable)	10.633	13.09	1633 \pm 5.0%	No Exp.	5879 10^{-16}	
⁵⁹ Co(n,2n) ⁵⁸ Co (ε , 70 d) \rightarrow ⁵⁸ Fe (stable)	10.633	13.09	410 \pm 0.0%	405 \pm 2.5%	1476 10^{-16}	
²³⁸ U(n,3n) ²³⁶ U (α , $2.34 \cdot 10^7 \text{ y}$) \rightarrow ²³² Th (stable)	11.330		163 (B-VII.1)	No Exp.	567 10^{-16}	AMS threshold ^W = 10^{-11}
⁵⁶ Fe(n,2n) ⁵⁵ Fe (ε , 2.74 y) \rightarrow ⁵⁵ Mn (stable)	11.40		170 (B-VII.1)	No Exp.	612 10^{-16}	AMS threshold ^W = 10^{-14}
⁸⁹ Y(n,2n) ⁸⁸ Y (ε , 107 d) \rightarrow ⁸⁸ Sr (stable)	11.612	13.90	346 \pm 1.3%	No Exp.	1246 10^{-16}	
⁵² Cr(n,2n) ⁵¹ Cr (ε , 27.7 d) \rightarrow ⁵¹ V (stable)	12.272	14.71	97 \pm 2.7%	No Exp.	360 10^{-16}	
²³ Na(n,2n) ²² Na (ε , 2.60 y) \rightarrow ²² Ne (stable)	12.419	15.40	8.6 \pm 1.2%	No Exp.	31 10^{-16}	
⁴⁶Ti(n,2n)⁴⁵Ti (ε, 3.1 h) \rightarrow ⁴⁵Sc (stable)	13.479	16.03	12.2 \pm 3.1%	93 \pm 33% (?)	44 10^{-16}	C/E = $0.13 \pm 33\%$???!!!
²⁷ Al(n,2n) ²⁶ Al (ε , $7.17 \cdot 10^5 \text{ y}$) \rightarrow ²⁶ Mg (stable)	13.55		5.7 (B-VII.1)	No Exp.	21 10^{-16}	AMS threshold ^W = 10^{-13}
⁵⁴ Fe(n,2n) ⁵³ Fe (ε , 8.5 min) \rightarrow ⁵³ Mn (3.7 My)	13.629	16.48	3.5 \pm 1.5%	No Exp.	13 10^{-16}	not for AMS ^W due to impact of ⁵⁴ Fe(n,np+d) ⁵³ Mn

IRDFF reactions and their products	E _{thr} MeV	E _{50%} MeV	SPA, μb		N _{product} / N _{target} if $10^8 \text{n/cm}^2/\text{s}$, 1000h	Comments
			IRDFF ¹	Experiment ²		
²⁵²Cf(s.f.) Spontaneous Fission Spectra: given ²⁵² Cf produces Flux = $10^8 \text{n/cm}^2/\text{s}$ (i.e. at $\approx 1 \text{ cm}$ from ²⁵² Cf of 10^9n/s intensity ³) and Irradiation of sample = 1000 h = 4.17 weeks						
²⁰⁹ Bi(n,3n) ²⁰⁷ Bi (ε , 31.6 y) \rightarrow ²⁰⁷ Pb (stable)	14.416	18.21	19 \pm 6.0%	No Exp.	$68 \cdot 10^{-16}$	
¹⁶⁹ Tm(n,3n) ¹⁶⁷ Tm (ε , 9.3 d) \rightarrow ¹⁶⁷ Er (stable)	14.963	18.49	14.7 \pm 5.7%	No Exp.	$54 \cdot 10^{-16}$	
⁵⁹ Co(n,3n) ⁵⁷ Co (ε , 271 d) \rightarrow ⁵⁷ Fe (stable)	19.352	22.36	0.097 \pm 5.6%	No Exp.	$0.35 \cdot 10^{-16}$	

Example of calculation for ²⁷Al(n,2n)²⁶Al: Ratio ²⁶Al/²⁷Al = Flux \times Time \times Sigma = $1.E+8 \text{n/cm}^2/\text{s} \times 3.6E+6 \text{s} \times 5.7E-30 \text{cm}^2 = 20.5E-16$

Table 2. Dosimetry reactions, their stable products, kinematic threshold E_{thr}, effective energy E_{50%} and SPA in the ²³⁵U(n_{th,f}) field, sorted by increasing E_{50%}.

IRDFF reactions and their products	E _{thr} MeV	E _{50%} MeV	SPA, μb		N _{product} / N _{target} if $10^9 \text{n/cm}^2/\text{s}$, 100 h	Comments
			IRDFF ¹	Experiment ²		
²³⁵U(n_{th,f}) neutron induced Fission Spectra: given n-Source produce Flux = $10^9 \text{n/cm}^2/\text{s}$ (cp. $1.9 \cdot 10^9 \text{n/cm}^2/\text{s}$ from fission plate in KUR facility ⁴) and Irradiation of sample = 100 h = 0.417 weeks						
¹⁶⁹ Tm(n,2n) ¹⁶⁸ Tm (ε , 93 d) \rightarrow ¹⁶⁸ Er (stable)	8.082	10.40	$3744 \pm 2.6\%$	$3735 \pm 4.2\%$	$13478 \cdot 10^{-16}$	
¹¹⁵ In(n,2n) ¹¹⁴ In (IT, 50 d; β^-) \rightarrow ¹¹⁴ Sn (stable)	10.633	11.60	$861 \pm 5.5\%$	No Exp.	$3100 \cdot 10^{-16}$	
¹⁴¹ Pr(n,2n) ¹⁴⁰ Pr (ε , 3.4 min) \rightarrow ¹⁴⁰ Ce (stable)	9.464	11.65	$1043 \pm 12.0\%$	No Exp.	$3755 \cdot 10^{-16}$	
⁶⁵ Cu(n,2n) ⁶⁴ Cu (ε , 12.7 h) \rightarrow ⁶⁴ Ni (stable) ⁶⁴ Cu (β^- , 12.7 h) \rightarrow ⁶⁴ Zn (stable)	10.065	12.46	$318 \pm 2.0\%$	No Exp.	$^{64}\text{Ni}/^{65}\text{Cu} = 704 \cdot 10^{-16}$ $^{64}\text{Zn}/^{65}\text{Cu} = 441 \cdot 10^{-16}$	
⁷⁵ As(n,2n) ⁷⁴ As (ε , 17.8 d) \rightarrow ⁷⁴ Ge (stable)	10.383	12.70	$295 \pm 6.4\%$	No Exp.	$1062 \cdot 10^{-16}$	
⁵⁹ Co(n,2n) ⁵⁸ Co (ε , 70 d) \rightarrow ⁵⁸ Fe (stable)	10.633	13.09	$191 \pm 1.8\%$	$203 \pm 2.5\%$	$688 \cdot 10^{-16}$	
²³⁸ U(n,3n) ²³⁶ U (α , $2.34 \cdot 10^7$ y) \rightarrow ²³² Th (stable)	11.330		682 (BVII.0)	No Exp.	$2455 \cdot 10^{-16}$	
⁵⁶ Fe(n,2n) ⁵⁵ Fe (ε , 2.74 y) \rightarrow ⁵⁵ Mn (stable)	11.400		739 (BVII.1)	No Exp.	$2660 \cdot 10^{-16}$	AMS threshold ^W = 10^{-14}
⁸⁹ Y(n,2n) ⁸⁸ Y (ε , 107 d) \rightarrow ⁸⁸ Sr (stable)	11.612	13.90	$149 \pm 1.4\%$	$150 \pm 3.3\%$	$536 \cdot 10^{-16}$	

IRDFF reactions and their products	E _{thr} MeV	E _{50%} MeV	SPA, μb		N _{product} / N _{target} if $10^9 \text{n/cm}^2/\text{s}$, 100 h	Comments
			IRDFF ¹	Experiment ²		
$^{235}\text{U(n}_{\text{th}},\text{f})$ neutron induced Fission Spectra: given n-Source produce Flux = $10^9 \text{n/cm}^2/\text{s}$ (cp. $1.9 \cdot 10^9 \text{n/cm}^2/\text{s}$ from fission plate in KUR facility⁴) and Irradiation of sample = 100 h = 0.417 weeks						
$^{52}\text{Cr(n,2n)}^{51}\text{Cr} (\varepsilon, 27.7 \text{ d}) \rightarrow ^{51}\text{V} (\text{stable})$	12.272	14.71	$38 \pm 2.7\%$	No Exp.	$137 \cdot 10^{-16}$	
$^{23}\text{Na(n,2n)}^{22}\text{Na} (\varepsilon, 2.60 \text{ y}) \rightarrow ^{22}\text{Ne} (\text{stable})$	12.419	15.40	$3.2 \pm 1.3\%$	No Exp.	$12 \cdot 10^{-16}$	
$^{46}\text{Ti(n,2n)}^{45}\text{Ti} (\varepsilon, 3.1 \text{ h}) \rightarrow ^{45}\text{Sc} (\text{stable})$	13.479	15.81	$4.3 \pm 4.4\%$	No Exp.	$15 \cdot 10^{-16}$	
$^{27}\text{Al(n,2n)}^{26}\text{Al} (\varepsilon, 7.17 \cdot 10^5 \text{ y}) \rightarrow ^{26}\text{Mg} (\text{stable})$	13.550		<i>2.0 (BVI.1)</i>	No Exp.	$7 \cdot 10^{-16}$	AMS threshold ^W = 10^{-13}
$^{54}\text{Fe(n,2n)}^{53}\text{Fe} (\varepsilon, 8.5 \text{ min}) \rightarrow ^{53}\text{Mn} (3.7 \text{ My})$	13.629	16.48	$1.2 \pm 5.1\%$	No Exp.	$4 \cdot 10^{-16}$	not for AMS ^W due to impact of $^{54}\text{Fe(n,np+d)}^{53}\text{Mn}$
$^{209}\text{Bi(n,3n)}^{207}\text{Bi} (\varepsilon, 31.6 \text{ y}) \rightarrow ^{207}\text{Pb} (\text{stable})$	17.416	17.88	$5.4 \pm 5.9\%$	No Exp.	$19 \cdot 10^{-16}$	
$^{169}\text{Tm(n,3n)}^{167}\text{Tm} (\varepsilon, 9.3 \text{ d}) \rightarrow ^{167}\text{Er} (\text{stable})$	14.963	18.20	$4 \pm 6.1\%$	No Exp.	$14 \cdot 10^{-16}$	
$^{59}\text{Co(n,3n)}^{57}\text{Co} (\varepsilon, 271 \text{ d}) \rightarrow ^{57}\text{Fe} (\text{stable})$	19.352	21.92	$0.017 \pm 7.7\%$	No Exp.	$0.06 \cdot 10^{-16}$	

Example of calculation for $^{27}\text{Al(n,2n)}^{26}\text{Al}$: Ratio $^{26}\text{Al}/^{27}\text{Al}$ = Flux \times Time \times Sigma = $1.E+9 \text{n/cm}^2/\text{s} \times 3.6E+5 \text{s} \times 2.E-30 \text{ cm}^2 = 7.2E-16$

Comments for Tables 1 and 2:

Italic font - reactions currently not included in IRDFF

1) Calculated SPA uncertainty includes only IRDFF-1.05 cross section uncertainty.

2) The known measurements are carried out by activation technique.

3) The most intensive ^{252}Cf sources known up to now:

K. Kobayashi et al. [JNST 19(1982)341] used 500 μg of ^{252}Cf which produced $\approx 1 \cdot 10^9 \text{n/s}$;

J. Czikai et al. [Antwerp (1982)418] used 40 μg (?) of ^{252}Cf which produced $\approx 1 \cdot 10^8 \text{n/s}$ (given 1 $\mu\text{g} = 2.3 \cdot 10^6 \text{n/s}$);;

M. Blinov et al. [Atom. Energiya 65(1988)206] used 2-3 Cf sources of 18 - 50 μg total mass or $0.4 - 1.2 \cdot 10^8 \text{n/s}$ (given 1 $\mu\text{g} = 2.3 \cdot 10^6 \text{n/s}$);

4) The most intensive PFNS source:

KUR power fission plate: $\varnothing 27 \times 1 \text{ cm}$, 1.1 kg of 90% ^{235}U , incident thermal n-flux = $5.8 \cdot 10^8 \text{n/cm}^2$ [I. Kimura and K. Kobayashi NSE106(1990)332]

W) - information from private communication with A. Wallner

N_{product} / N_{target} - looks to be feasible for AMS

For other high energy reactions see: Cf-252(s.f.) http://www-nds.iaea.org/IRDFFtest/IRDFF105_MCNP_Cf.pdf
U-235(n_{th},f) http://www-nds.iaea.org/IRDFFtest/IRDFF_MCNPtest_U5.pdf.

Tables 1 and 2 show that it was impossible to measure so far some high threshold SPA by traditional activation technique with SPA below 150 - 400 μ b. SPA for these reactions, if they can be measured by activation or alternative methods, will probe the unknown high energy part (i.e. above 8-10 MeV where uncertainties \approx 100%) of the ²⁵²Cf(s.f.) and ²³⁵U(n,f) spectra, since the dosimetry and some other reaction cross sections are known there with much better accuracy (\leq 10%).

II. Techniques alternative to Activation

1. The Accelerator Mass Spectrometry (AMS) was shown is feasible to measure extremely small SPA.

The method sensitivity $N_{\text{product}} / N_{\text{target}} \sim 10^{-12} - 10^{-16}$.

For more details see A. Wallner et al.:

“Novel method to study neutron capture of ²³⁵U and ²³⁸U simultaneously at keV energies”, [Phys. Rev Lett. 112\(2014\)192501](#)

“Precise measurement of the ²⁷Al(n,2n)^{26g}Al excitation function near threshold and its relevance for fusion-plasma technology”, [J. Eur. Phys. A7, 285 \(2003\)](#)

“Production of Long-lived Radionuclides ¹⁰Be, ¹⁴C, ⁵³Mn, ⁵⁵Fe, ⁵⁹Ni and ^{202g}Pb in a Fusion Environment” [J. Korean Phys. Soc. 59, 1378](#)

“Nuclear Data from AMS & Nuclear Data for AMS – some examples”, [EPJ 35 \(2012\) 01003](#)

“Accelerator Mass Spectrometry & Neutron-induced Reactions”, presentation at the IAEA TM on Standards (July 2013) [here](#).

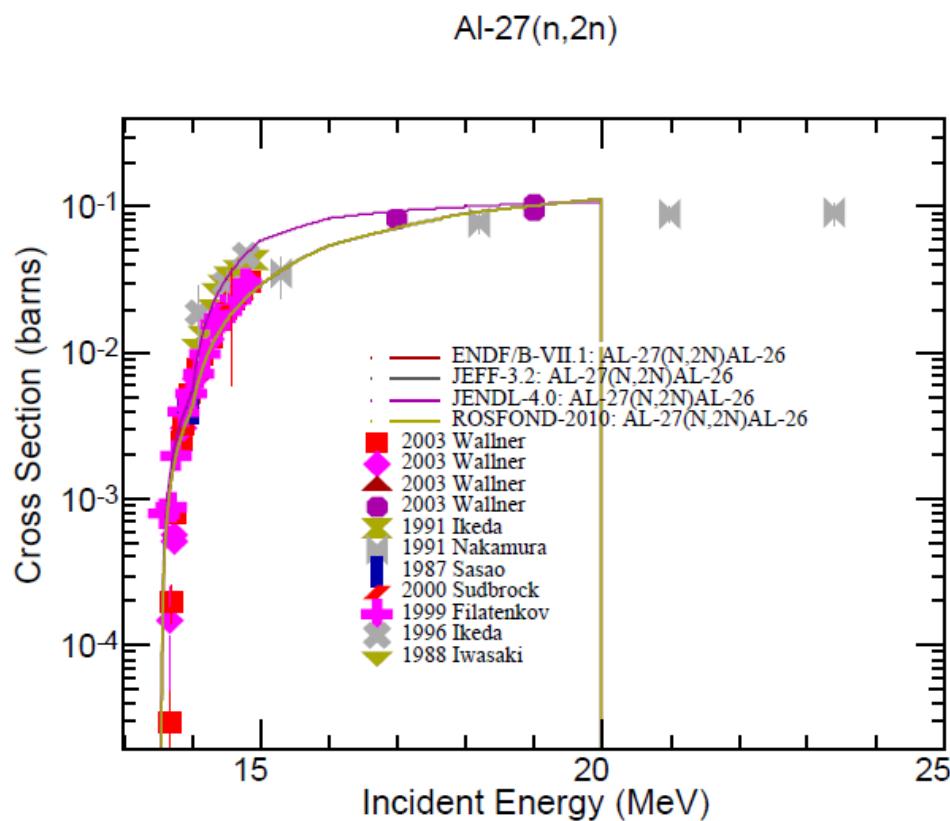
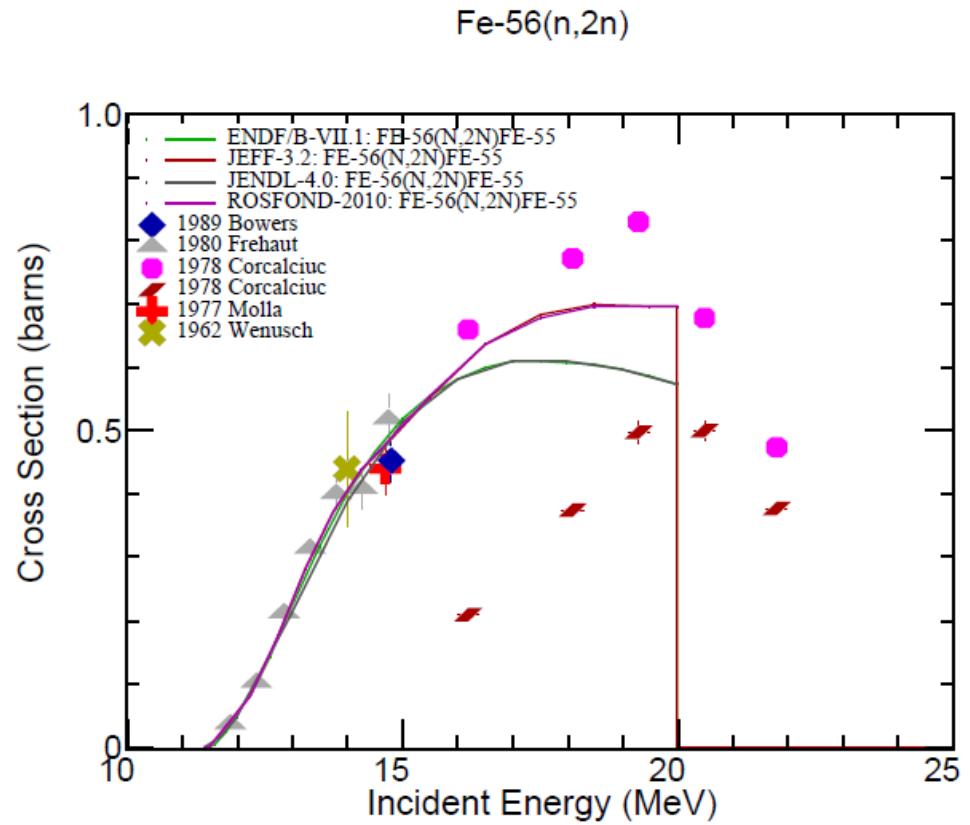
A. Wallner pointed out on the following **high threshold non-dosimetry reactions accessible for AMS:**

²⁷Al(n,2n)²⁶Al was measured by AMS up to 19 MeV with accuracy 10% by A. Wallner et al., [Eur. Phys. A17, 285 \(2003\)](#))

⁵⁶Fe(n,2n)⁵⁵Fe was measured by AMS around 14 MeV by A. Wallner et al. [J. Korean Phys. Soc. 59, 1378](#));

²³⁸U(n,3n)²³⁶U was measured by AMS at 14 MeV by X. Wang et al. [Phys. Rev. C87\(2013\)014612](#)).

The status of these reaction cross sections are shown in Figs. 3-5.

Fig 3. Available experimental and evaluated data for $^{27}\text{Al}(\text{n},2\text{n})^{26}\text{Al}$.Fig. 4. Available experimental and evaluated data for $^{56}\text{Fe}(\text{n},2\text{n})^{55}\text{Fe}$.

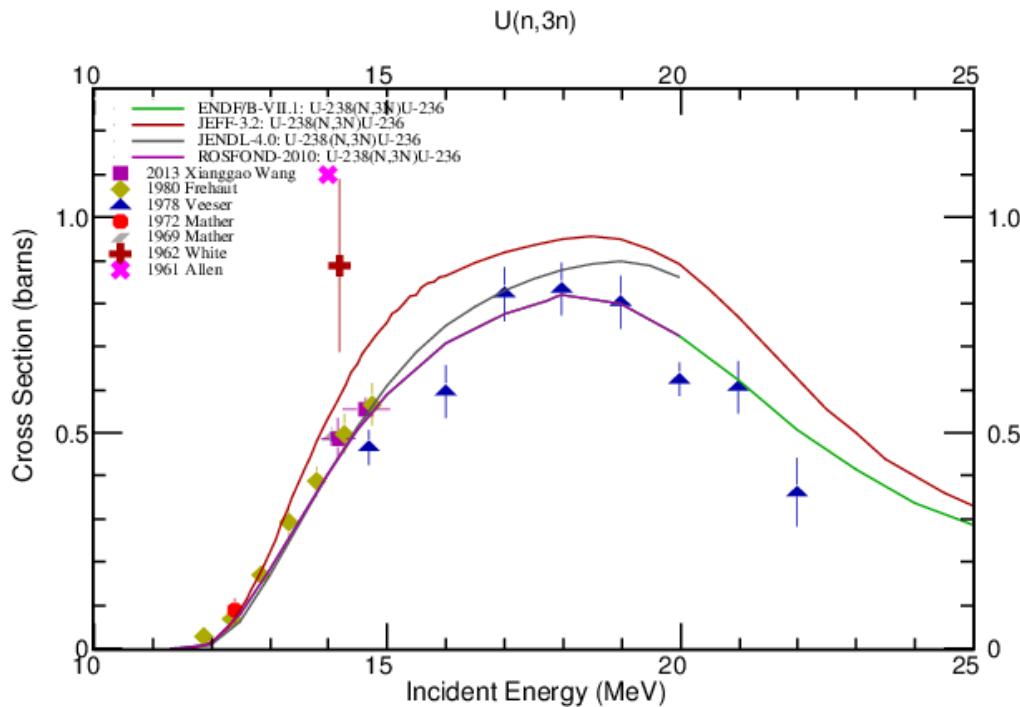


Fig. 5. Available experimental and evaluated data for $^{238}\text{U}(\text{n},3\text{n})^{236}\text{U}$.

2. Prompt Gamma Neutron Activation Analysis (PGNAA)

The method sensitivity $\text{N}_{\text{product}} / \text{N}_{\text{target}} \sim 100 \text{ ppm} = 10^4$.

This technique was proved is capable to measure the non- threshold SPA cross sections

by employing the PGNAA facility of FRM-II after Ni foil irradiation in the LVR-15 reactor (fluence rate $3.10^{14} \text{ cm}^{-2}\text{s}^{-1}$)
for reactions $^{62}\text{Ni}(\text{n},\gamma)^{63}\text{Ni}$ ($T_{1/2} = 101.2 \text{ y}$, Atlas $\sigma(n_{\text{thermal}},\gamma) = 14.9 \text{ b}$) and $^{58}\text{Ni}(\text{n},\gamma)^{59}\text{Ni}$ ($T_{1/2} = 7.6 \cdot 10^4 \text{ y}$, Atlas $\sigma(n_{\text{thermal}},\gamma) = 4.37 \text{ b}$).

For principles, first results and publications see:

V. Klupák, L. Viererbl, Z. Lahodová, J. Šoltés, I. Tomandl, P. Kudějová, “Nickel foil as transmutation detector for neutron fluence measurements”, [ISRD-15, EPJ Web of Conferences 106, 05013 \(2016\)](#).

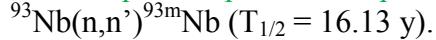
I. Tomandl, L. Viererbl, P. Kudějová, Z. Lahodová, V. Klupák, M. Fikrle, “Determination of trace concentrations of transmuted stable nuclides in TMD detectors using PGAA”, [J. of Radioanal. and Nuclear Chemistry, 300 \(2014\) 1141](#).

3. Resonance Ionization Mass Spectroscopy (RIMS) - Isotope measurements based on Laser Spectroscopy.

The method sensitivity $N_{\text{product}} / N_{\text{target}} \sim ??$.

Currently under development for the trace analysis of short-lived and long-lived radioactive nuclei.

This technique was proved is capable to measure the cross section for dosimetry reaction



For principles, first results and publications see:

here: http://coe.nucl.nagoya-u.ac.jp/Measurement01_E.html and

H. Tomita, T. Takatsuka, T. Iguchi, Y. Adachi, Y. Furuta, T. Takamatsu, T. Noto, "Development of Neutron Dosimetry Technique with $^{93}\text{Nb}(\text{n},\text{n}')^{93m}\text{Nb}$ Reaction by Resonance Ionization Mass Spectrometry", [ISRD-15, EPJ 106, 05002 \(2016\)](#)

T. Takatsuka, H. Tomita, V. Sonnenschein, T. Sonoda, Y. Adachi et al. "Development of resonance ionization in a supersonic gas-jet for studies of short-lived and long-lived radioactive nuclei", [NIM B 317 \(2013\)586](#)

4. Ion Beam Analysis (IBA) technique such as PIXE, PIGE etc.

The method sensitivity $N_{\text{product}} / N_{\text{target}} \sim 10^{-4} - 10^{-3}$.

It seems will not be possible to use this technique to measure the high threshold SPA cross sections ($< 1 \text{ mb}$) because of its low sensitivity however it may work, as PGNAA, for the non- threshold reactions with large SPA cross sections ($> 1 \text{ b}$).

5. Nuclear magnetic resonance (??).

The method sensitivity $N_{\text{product}} / N_{\text{target}} \sim ??$.